

Evaluation of Indoor Radiation Hazard on Worker & Public Health in Mitford Hospital, Dhaka, Bangladesh

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ABSTRACT **ARTICLE INFO** Received: 15 Jun. 2020 Objective: Radiation workers in nuclear Medicine institute are handling unsealed radioactive materials for diagnostic and therapeutic procedures of patients and thus radiation hazard on workers and public health in Accepted: 30 Jul. 2020 nuclear medicine is high comparing to other departments of the hospital. The purpose of the study is to evaluate the radiation hazard on workers and public at the indoor places of the Institute of Nuclear Medicine & Allied Sciences (INMAS) Mitford, Sir Salimullah Medical College and Hospital Campus based on the real-time radiation monitoring data. Methods: The radiation monitoring was performed using a real-time portable digital radiation monitoring device. This real-time digital portable radiation monitoring device meets all European CE standards as well as the American "FCC 15 standard". The portable digital radiation monitoring device was placed at 1 meter above the ground on tripod and data acquisition time for each monitoring point (MP) was 1 hour. 24 MPs were selected for collection of radiation dose rates at different indoor locations of INMAS, Mitford hospital from May-June 2019. The real-time dose rate also monitored at 1 meter distance from injected patients in the patient's waiting room after injecting 99mTc & 131I. Results: The measured dose rates were ranged from 0.181 ± 0.057 μ Sv.h-1 to 2.247 ± 0.685 μ Sv.h-1 with an average of 0.463 ± 0.695 μ Sv.h-1. The annual effective dose to the radiation worker and public were varied from 0.279 ± 0.089 mSv to 3.481±1.061 mSv with an average of 0.717 ±1.077mSv. Excess life-time cancer risk (ELCR) of worker and public were evaluated based on annual effective dose and varied from 1.113 X 10-3 to 1.385 X 10-2. Conclusion: Real-time radiation monitoring at indoor places of nuclear medicine facilities are required for detection of contamination in the workplace. So this study is needed to keep the indoor environment free from radiation hazard and thereby improving the worker and public health.

Keywords: indoor, ionizing radiation, nuclear medicine, in-situ, occupational health

INTRODUCTION

Ionizing radiation is being widely used in the hospital for diagnostic and therapeutic procedures of patients. United Nations Scientific Committee on the Effects of Atomic Radiation (UNSCEAR, 2008) estimated that there are about four billion radiographic examinations performed each year by medical Staff using ionizing radiation for diagnostic and therapeutic purposes involving to radiation worker and public health hazard if radiation protection & safety guidelines are not properly practicing (Fazel et al., 2009). Radiation worker in the nuclear medicine institutes are always receiving small amount of ionizing radiation in the workplace while handling radioactive substances in spite of using personal radiation protective equipments (Dobrzyn´ska et al., 2014; Fazel et al., 2009). The origin of the natural radioactivity of construction materials to be known for the estimation of population exposure to radiation, because the majority of the population spends approximately 80% of their time in the indoor environment (UNSCEAR, 2008). Radiation worker in the nuclear medicine institute used to handle unsealed radioactive substances for diagnostic and therapeutic procedures of patients. Hence, the probability of getting contamination in the indoor environment of the nuclear medicine institute is more than other departments of the hospital. Overexposure and uncontrolled exposure to ionizing radiation are most important factors causing cancer and genetic mutations (Borgen et al., 2014; Bouraoui et al., 2013; Hricak et al., 2011). So, real-time radiation monitoring in the indoor environment during working time is very important for minimizing the contamination in the indoor environment and thereby keeping the radiation dose to worker and public as low as possible. The medical applications of ionizing radiation, while offering great benefit to patients, also contribute significantly to radiation exposure of worker and public (UNSCEAR, 2000; EURATOM,

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1997; UNSCEAR, 1993). Radiation worker and public exposure to ionizing radiation due to diagnostic and therapeutic procedures has increased sharply in recent years (NCRP, 2009; UNSCEAR, 2008). Among the medical staff, those primarily getting more radiation exposure are nuclear medicine staff. Radiation worker in the nuclear medicine institute used to handle unsealed radioactive substances that contribute to external and internal radiation exposure to radiation worker. The quantity of radiation exposure to worker depends on radionuclide, activity and type of procedure within a department. Significant number of medical procedures involving beta particle emitting radionuclides, extremity e.g., fingers of hand exposures and probable skin contamination of nuclear medicine worker is great concern. While performing clinical nuclear medicine procedures, the quantity of radiation exposure to worker depends on the proper handling of the radioactive substances, e.g., proper wearing of the personnel protective equipments (PPEs) namely, lead apron, hand gloves, lead glass, socks, shoes, etc. and the proper syringe shields to be used when administering radiopharmaceuticals. Radiation worker (Technologists, Technician, nurses) have to be close contact with the patient during administering radiopharmaceuticals, positioning the patient and the camera. Generally, the imaging procedures contribute the highest radiation exposure to worker (Barrall et al., 1976). Internal radiation exposures to worker are much lower than external radiation exposures and are reduced by monitoring working surfaces and airborne concentrations (NCRP, 1990). Due to the possibility of getting internal radiation exposure, higher values of annual effective dose are anticipated for worker preparation involving in the and assay of radiopharmaceuticals than for medical doctors and nurses in the nuclear medicine institute. The meaning of radiation monitoring is to control the dose accumulation pattern of individual (UNSCEAR, 1982) includes a programme of measurements, estimations and record keeping of radiation exposure to worker. The aim of the present study is to evaluate the radiation hazard on worker and public health based on the real-time radiation monitoring data of INMAS Mitford Hospital following In-Situ method. Real-time radiation monitoring at indoor environment of the nuclear medicine institute is crucial for minimizing the radiation hazard on worker & public and thereby to keep the radiation dose to worker and public as low as possible.

MATERIALS AND METHODS

Description of the Equipment

A real-time DPRMD was used for this study. The DPRMD is German designed and manufactured, built with a solid Novadur exterior. An optional stylish leather holster with belt strap can further protect the DPRMD from the elements. The DPRMD meets all European CE standards as well as US FCC 15. All units come with an industry leading 2-year manufacturer's warranty and a serialized test certificate. The DPRMD is a fully featured Geiger counter with a form fitting ergonomic shape. The unit has a battery indicator, multiple unit conversion, real-time dose rate and cumulative dose display functions and programmable logging and alert functions. Advanced functions include PC data download via USB cable and an ultra low current power circuit for extended battery life. The DPRMD accurately measures dose rate within the range of $0.01-5000 \mu$ Sv/hr (User Manual-GAMMA SCOUT, 2014).

Calibration of the Equipment

The DPRMD was calibrated inbuilt by the manufacturer. The DPRMD is also calibrated using the gamma-ray standard sources such as ¹³⁷Cs, ⁶⁰Co, etc. and X-ray Unit at the Secondary Standard Dosimetry Laboratory (SSDL) under the Bangladesh Atomic Energy Commission (BAEC). The SSDL of BAEC has been available since 1991, which is traceable to the Primary Standard Dosimetry Laboratory (PSDL) of National Physical Laboratory (NPL), UK. The SSDL of BAEC has X-ray Unit (30 kV-225 kV) which is needed for calibration of the radiation monitoring equipments. The performance of BAEC's SSDL has been kept as per requirements of the International Atomic Energy Agency (IAEA)/World Health Organization (WHO) network of SSDLs. Hence, the real-time radiation monitoring data of DPRMD meets up the International monitoring system.

Description of the Monitoring Site

The MPs were marked out using GARMIN eTrex HC series personal navigator. The unit uses the proven performance of Garmin high-sensitivity GPS and full-featured mapping to create an unsurpassed portable GPS receiver (Owner's Manual-GARMIN eTrex HC Series, 2007). The GPS reading of the MPs were ranged from E: 90°40.022' to E: 090°40.108' and from N: 23°71.206' to N: 23°71.283'. Twenty four MPs were selected in the indoor environment for collection of the real-time radiation dose rates at the INMAS Mitford, Sir Salimullah Medical College and Hospital Campus following In-Situ Method. The MPs include hot lab, thyroid lab, gamma camera lab/SPECT-CT lab, RIA lab, patient waiting rooms, visitor waiting rooms, common spaces, corridors, etc. The real-time radiation monitoring was performed from May-June 2019 using the DPRMD. For each MP, the real-time DPRMD was placed on tripod at 1 m height and the real-time dose rate monitoring time was 1 hour. The brief description of the MPs are depicted in Table 1. The MPs were marked out using Global Positioning System (GPS) navigation. INMAS Mitford Hospital is an institute of Bangladesh Atomic Energy Commission (BAEC) under the Ministry of Science & Technology, Government of Bangladesh. The INMAS Mitford hospital is situated in the campus of Sir Salimullah Medical College and Mitford Hospital which is the oldest medical hospital in Dhaka and also in the country.

Annual Effective Dose and ELCR Estimation

The indoor occupancy factor of public is 0.80 (UNCEAR, 1988). This occupancy factor is the fraction of the total time during which a person is exposed to a radiation field at indoor. The indoor annual effective dose to public due to radiation is estimated according to the following equation:

Annual effective dose (
$$\mu Sv$$
)
= dose rate ($\mu Sv.hr^{-1}$) × 0.8 (1)
× 8760 hr.yr⁻¹

Excess life-time cancer risk (ELCR) is estimated using the following equation:

$$ELCR = AED \times DL \times RF \tag{2}$$

Where AED is the annual effective dose to radiation worker and public, DL is the duration of life of Bangladeshi people (http://en.worldstat.info/Asia/Bangladesh, 2019) and RF is the risk factor (Sv^{-1}), it is a fatal cancer risk per Sievert. For stochastic effects from low-dose radiation, ICRP 103 suggested the value of 0.057 for the public exposure (ICRP, 2007).

RESULTS AND DISCUSSION

The measured mean annual effective dose for radiation worker was 0.717 mSv which is lower than the worldwide average annual effective dose (1.4 mSv) for radiation worker in nuclear medicine (UNSCEAR, 2008). The maximum radiation

dose for one day in SPECT-CT room at INMAS Mitford hospital was 14.38 μ Sv which is lower than the maximum allowable radiation dose for one day (55 µSv) in nuclear medicine institute (ICRP, 2008). The mean annual effective dose for radiation worker at nuclear medicine institutes in Greece during the period 2000-2002 was ranged from 0.75-1.49 mSv (UNSCEAR, 2008). The mean annual effective dose to radiation worker for different indoor environment of INMAS Mitford hospital was remained within the radiation dose range of nuclear medicine institutes in Greece except SPECT-CT Lab. Taking the conversion factor of 0.7 Sv.Gv⁻¹ as suggested by UNSCEAR (UNSCEAR, 2000) and taking into account that public in Bangladesh spend about 20% of their time outdoor and remaining 80% of time indoor; the annual effective dose of worker and public of INMAS Mitford Hospital campus in Dhaka city were estimated and depicted in Table 1.

Table 1. Annual effective dose of worker and public due to radiation at indoor environment of INMAS Mitford Hospital

CI Ma	Name of Place	Latitude/	Latitude/ Radiation dose rate (µSv			hr) Annual effective dose due to		
51. NO.		Altitude	Range	Mean	SD	radiation (mSv) ± SD		
1.	Gamma camera lab	N23°42.765 E90°24.032	0.970-1.810	1.323	0.203	2.049 ± 0.315		
2.	Thyroid lab	N23°42.733 E90°24.023	0.840-2.600	1.502	0.389	2.327 ± 0.603		
3.	RIA lab	N23°42.733 E90°24.034	0.100-0.260	0.195	0.043	0.302 ± 0.068		
4.	SPECT-CT lab	N23°42.753 E90°24.038	0.169-3.360	2.247	0.685	3.481 ± 1.061		
5.	Visitor/patient waiting room-1 ground floor	N23°42.738 E90°24.018	0.100-0.510	0.289	0.108	0.449 ± 0.168		
6.	Visitor/patient waiting room-3 ground floor	N23°42.733 E90°24.023	0.130-4.730	0.798	1.339	1.235 ± 2.073		
7.	Ground floor Counter	N23°42.728 E90°24.012	0.050-0.320	0.212	0.056	0.328 ± 0.087		
8.	Ground space	N23°42.764 E90°24.028	0.070-0.300	0.181	0.057	0.279 ± 0.089		
9.	Ground floor South side	N23°42.723 E90°24.028	0.180-4.710	1.426	1.666	2.208 ± 2.581		
10.	Visitor/patient waiting room-1 1 st floor	N23°42.747 E90°24.025	0.120-0.250	0.189	0.033	0.292 ± 0.052		
11.	Visitor/patient waiting room-2 1st floor	N23°42.739 E90°24.038	0.130-0.270	0.205	0.035	0.318 ± 0.054		
12.	Visitor/patient waiting room-3 1st floor	N23°42.739 E90°24.014	0.020-0.320	0.192	0.064	0.298 ± 0.099		
13.	1 st floor south side	N23°42.739 E90°24.026	0.100-0.280	0.200	0.045	0.310 ± 0.071		
14.	1 st floor north side	N23°42.764 E90°24.041	0.130-0.260	0.196	0.037	0.304 ± 0.057		
15.	1 st floor common space	N23°42.729 E90°24.023	0.070-0.310	0.221	0.054	0.343 ± 0.084		
16.	2 nd floor south side	N23°42.728 E90°24.025	0.050-0.360	0.203	0.069	0.315 ± 0.107		
17.	2 nd floor north side	N23°42.733 E90°24.038	0.100-0.270	0.188	0.045	0.291 ± 0.071		
18.	2 nd floor common space	N23°42.729 E90°24.025	0.070-0.270	0.193	0.051	0.299 ± 0.080		
19.	1 st floor common space	N23°42.746 E90°24.030	0.100-0.280	0.218	0.041	0.337 ± 0.064		
20.	3 rd floor north side	N23°42.729 E90°24.027	0.090-0.300	0.198	0.055	0.307 ± 0.086		
21.	3 rd floor south side	N23°42.728 E90°24.025	0.130-0.250	0.185	0.033	0.287 ± 0.052		
22.	3 rd floor east side	N23°42.734 E90°24.024	0.050-0.270	0.189	0.052	0.292 ± 0.082		
23.	Common space 3 rd floor	N23°42.733 E90°24.024	0.120-0.270	0.194	0.045	0.301 ± 0.069		
24.	Common space 3 rd floor	N23°42.745 E90°24.033	0.140-0.290	0.215	0.046	0.333 ± 0.071		

Name of indoor place of	Range	Mean dose rate	Annual range/Mean annual	Country	Doforonco
hospital	(µSv/hr)	(µSv/hr)	effective dose (mSv)	Country	Reference
Kwali General Hospital	0.100-0.124	0.107 ± 0.003	0.750 ± 0.020	Nigeria	James et al. (2015)
Skane radiodiagnostic centre	-	-	2.063	Nigeria	Jwanbot et al. (2012)
Plateau state specialized	-	-	2.443	Nigeria	Jwanbot et al. (2012)
blattiwatte ineliiofiai	0.140-0.160	0.146 ± 0.02	_	Nigeria	Okoye and Avwiri (2013)
dopartment)			-		
Braithwaite memorial					
specialized hospital (in wards)	0.12-0.19	0.136 ± 0.02	-	Nigeria	Okoye and Avwiri (2013)
Woliso hospital	0.10-0.15				
(control room waiting room x-	0.136-0.141	0.136 ± 0.01	_	Ethionia	Geletu et al. (2017)
ray room)	0.13-0.147	0.150 0.01		Ethopia	Geleta et al. (2017)
Butaiera hospital	0.07-0.13:				
(control room:waiting room: x-	0.12-0.15:	0.14 ± 0.02	-	Ethiopia	Geletu et al. (2017)
rav room)	0.105-0.172				
Mercy hospital	0.033-0.017;				
(control room; waiting room; x-	0.10-0.10;	0.14 ± 0.01	-	Ethiopia	Geletu et al. (2017)
ray room)	0.10-0.15				
Atat hospital	0.117-0.158;				
(control room; waiting room;x-	0.08-0.13;	0.148 ± 0.03	-	Ethiopia	Geletu et al. (2017)
ray room)	0.13-0.85			_	
Gaza Strip hospital			5.9-14.2;		
CT scan room control panel;	-	-	6.3-12.3;	Palestine	Abu Zer et al. (2016)
corridors; patient waiting room			0.8-13.2		
Teaching Sohag Hospital Linear	0.094-4.30;		0.659-30.134;		
accelerator:CT:Y_ray	0.084-12.528;	0.10 ± 0.023	0.589-87.796;	Egypt	Harb (2016)
accelerator, G1, A-ray	0.076-0.15		0.533-1.051		
INMAS Mitford Hospital			0.031-7.32/0.518±0.988;	Bangladesh	This study
Visitor&Patient waiting room.	0.02-4.73; 0.05-4.71	0.335± 0.638; 0.282 ± 525	0.077-7.29/		
common space/corridors			0.436 ± 0.813		

Table 2. Comparison of dose rate and annual effective dose in the indoor environment of INMAS Mitford Hospital with other countries

Table 2 shows the indoor dose rate and annual effective dose of worker and public of INMAS Mitford Hospital and those values are compared with other countries. From **Table 2**, it can be seen that the real-time radiation dose rate and estimated annual effective dose at CT room of INMAS Mitford Hospital is lower than those of Teaching Sohag Hospital in Egypt (Harb, 2016) and Gaza Strip Hospital in Palestine (Abu Zer et al., 2016).

The amount of radioisotopes are injected into the adult patients for gamma camera or Computed Tomography (CT) imaging: for bone scan 10-20 mCi^{99m}Tc, for DTPA 3-5 mCi99mTc, for DMSA 4-5 mCi99mTc, for thyroid uptake 5-10 mCi131I and for thyroid scan 2 mCi99mTc. Real-time radiation dose rates of patient's waiting room were measured at 1 meter distance from the radioisotope injecting patients in the INMAS Mitford Hospital from May-June 2019. The radiation dose rates were monitored using the DPRMD. The measured dose rates were ranged from 13.420 µSv.h⁻¹ to17.690µSv.h⁻¹ with an average of 15.182± 1.169 µSv.h⁻¹. The radiation dose rate at 1 meter distance from the injecting patients who are waiting after injecting radioisotope in the waiting room for gamma camera/CT scan image (e.g., bone scan, thyroid scan, thyroid uptake) are comparable with those values of Pakistan (Javed et al., 2017) and Norway (Stenstad et al., 2014). The maximum allowable radiation dose of worker in a working day at the nuclear medicine institute is 55µSv (ICRP, 2008). The maximum radiation dose at 1 meter distance from radioisotopes injecting patients in a working day in the INMAS Mitford was calculated and it was found to be 141.52 μ Sv. From this study, it was observed that hospital staff and public entrance in the patient's waiting room after injection of radioisotopes have to be restricted in order to keep their radiation dose within the allowable limit. In addition to that radiation worker has to handle the radioactive substances and radiation generating equipments in the nuclear medicine institute as per the radiation protection and safety regulations of Bangladesh as well as international body recommendations during their daily work. **Figure 1** shows the frequency distribution of radiation dose rate at indoor environment of INMAS Mitford Hospital. From **Figure 1**, it is observed that the real-time radiation dose rate of 19 indoor locations remained within the range of 0.10-0.50 μ Sv/hr out of 24 indoor locations.

Figure 2 shows the mean annual effective dose in the indoor environment of each monitoring point was normalized to the minimum annual effective dose. From **Figure 2**, it can be seen that the difference of the mean annual effective dose of few monitoring locations are high. The reason for this high variation of mean annual effective dose for few indoor environment are that the different type of radiation generating equipments and different type of radioisotopes are handling/stored in the different laboratory of the INMAS Mitford Hospital.

The ELCR of medical staff and the public in the indoor environment of INMAS Mitford Hospital campus varied from



Figure 1. Frequency distribution of radiation dose rate at indoor environment of INMAS Mitford Hospital



Figure 2. Mean annual effective dose value of each MP normalized to the minimum annual effective dose in the indoor environment



Figure 3. ELCR of radiation worker and the public at indoor environment of INMAS Mitford hospital campus

 1.113×10^{-3} to 1.385×10^{-2} with an average value of 2.867×10^{-3} . The estimation of ELCR of medical staff and the public is based on the calculated annual effective dose in the indoor environment of various rooms and corridors/common spaces. From **Figure 3**, it can be seen that the variation of ELCR of medical staff and the public is high for few indoor locations. The reason is that radiation worker handles different type of radioactive substances and radiation generating equipments for diagnostic and therapeutic purposes that contributed to different exposure to worker and the public.

Usually, effective radiation dose for CT procedure is higher than other diagnostic imaging modalities (Wall and Hart, 1997) and this study found similar finding. It is mentioned in the Swedish Radiation Safety Authority report that CT and nuclear medicine took 16% of all radiological investigations except mammography and contributed to 64% of the collective radiation dose in Sweden in 2005 (Almen, A. et al., 2008). It is found in the National Council on Radiological Protection and Measurement in USA, CT and nuclear medicine took 22% of all radiological investigations but contributed to 75% of the collective radiation dose in US in 2006 (NCRP, 2009). Nevertheless, the radiation dose depends on the number of manipulations, the radioisotope types and the quantity of activity handling.

The mean excess life-time cancer risk (ELCR) of worker and public of INMAS Mitford Hospital is lower than the world average value of 5.57 X 10⁻³ (UNSCEAR, 2008) that need further research for verification. Even though the annual effective dose and ELCR values at indoor environment of INMAS Mitford hospital are lower than the world average value except SPECT-CT lab but all the values are much lower the radiation dose limit set for the radiation worker in the Nuclear Safety and Radiation Control Rules-1997 of Bangladesh (NSRC Rules, 1997). The estimated mean annual effective dose of 0.717 mSv is not expected to contribute considerable additional radiation hazard on worker and public health as per the radiological health hazard consideration. It may be mentioned here that the annual dose limit of public is 1 mSv and this dose limit is related to practices from planned exposure situations (e.g., nuclear installations or hospitals) and is not related to the radiation dose getting from existing exposure situations (e.g., natural sources of radiation) as per recommendations of the ICRP 103 (ICRP, 2007). The real-time radiation monitoring in the indoor environment of the hospital especially nuclear medicine institutes have to be performed regularly for the safety of the worker, public and the environment and keeping the indoor environment free from unnecessary radiation.

CONCLUSION

Medical staff used to handle unsealed radioactive substances and radiation generating equipments in the nuclear medicine institute for diagnostic and treatment purposes of patients. So, the probability of getting contamination in the indoor environment of the nuclear medicine institute is more than other departments of the hospital. For that reason, real-time radiation monitoring during working time in the indoor environment of the hospital is very important for the detection of contamination and consequently minimizing the radiation hazard on worker & public is possible and keeping the indoor environment of the hospital free from radiation hazard. The real-time measured dose rates in the indoor environment were ranged from 0.181 $\pm\,0.057\mu Sv.h^{-1}$ to $2.247\pm0.685\,\mu Sv.h^{-1}$ with an average of 0.463 \pm 0.695 µSv.h⁻¹. The estimated annual effective dose to the radiation worker and the public in the indoor environment were found to be in the range of 0.279 ± 0.089 mSv to $3.481\pm$ 1.061mSv with an average of 0.717 ± 1.077 mSv. Application of CT in medicine tremendously improved the medical imaging, but CT contributed more radiation exposure to radiation worker and the public. Hence, periodic education and training of radiation worker is required for proper handling of the radioactive substances as well as radiation generating equipments in the nuclear medicine institute in order to minimize the radiation hazard on worker and public health and to keep the indoor environment free from radiation hazard.

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